Simple Low Energy Gamma Ray Spectral Analysis Using Large NaI(Tl) Scintilation Spectrometers*

BY

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A simple method of conversion of pulse-height distributions measured with a large NaI(Tl) scintillator to energy spectra of gamma rays is described. This method is quite adequate for a low energy region where the peak-to-total ratio is large and for the application of approximate analysis in the outside field of the strict gamma ray spectroscopy. In practical application of this method one needs to know only the peak-to-total ratio and efficiency of the scintillation counter as a function of photon energy. Simulation by a calculation was carried out for the purpose of demonstrating the utility of this method.

Introduction

The need for a simple method of a coversion of the pulse height distribution measured by a NaI(Tl) scintillator to an energy spectrum of gamma rays become evident during the course of an investigation on the penetration and scattering of gamma rays.

The inverse matrix method^{1,2}, strip-off method¹ and least-squares method³ has been used for unfolding a scintillation pulse height distribution to the gamma ray energy spectrum. The strip-off method consists of successively subtracting the photopeaks with the corresponding tails step by step. The analysis using the least-spuares method has been applied as the progress of electronic computers inspite of the graphical strip-off. In most applications of the strip-off method and the least squares method, the response functions obtained by experiment or calculation with monochromatic gamma rays were used, and in a few works the photopeaks were approximated by delta functions or triangles having similar width of photopeaks $^{1,3-6}$. A large number of response functions of the scintillator for monochromatic gamma rays are necessary as input data to compose a response function matrix.

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Experimental determination of the response functions with desired monochromatic gamma rays is not easy, because a limited number of monochromatic gamma source is available and scattered gamma rays are incident to the scintillator. Calculations of response functions carried out with ideal conditions often disagreed with experimental values in low energy region.

Recently, large NaI(Tl) scintillators having very good characteristics are easily obtainable. The scintillators such as 8-in. diameter by 8-in. long have very large photofraction. The determination of the photofraction and pulse-height distribution for desired energy gamma rays is not easy; however, interpolation is easier for the photofraction than for the pulse-height distribution, if we can get them at several energies of gamma rays. Taking advantage of these facts, the simpler method of converting the pulse-height distributions to the gamma ray energy spectra, has been developed. It promises to be useful for the approximate spectral analysis of gamma rays in the outside field of strict gamma ray spectroscopy.

This paper describes a simple unfolding method* and a simulation by computation which demonstrates the utility of the method and several examples of application with a 8-in. diameter by 4-in. long NaI(Tl) scintillator.

Simplification of Unfolding

The simple unfolding method described in this paper is illustrated schematically in Fig. 1. Each unfolding strip has two parts. Part A, namely"peak", and part B, "tail", correspond to the photopeak and the Compton electron distribution in the usual strip-off method, respectively. The ratio, ("peak" area)/("peak" area + "tail" area), is made equal to the photofraction at the energy corresponding to the "peak" for the first approximation. The width of the "peak" is decide somewhat arbitrarily, keeping free from the pulse-height distribution or widths of photopeaks. It has to be determined considering the energy of the gamma rays, type of measuring instrument, and capacity of caluclating machine. In most cases, it is smaller than that of photopeak and has a constant value in all energy regions. In the usual strip-off method, on the contrary, the width of the photopeak is not arbitrary; it depends on the gamma ray energy and, worse, on the characteristics of the individual instrument.

A pulse-height distribution $P(\varepsilon)$ is sliced up first into pulse-height intervals which are made equal to the width of the "peak" of the unfolding strip and get the one-column matrix $\langle P(\varepsilon) \rangle$, where ε denotes pulse height. Starting from the

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^{*} The similar method was used independently by Nakata and described shortly in: M. Nakata, T. Fuse and K. Takeuchi, Unken Hokoku 11, 561–568 (1961) (in Japanese)

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Fig. 1. A schematic illustration of the simple unfolding method. A pulse-height distribution $P(\epsilon)$ is shown by the dark line. The areas A-I and A-II show examples of the "peaks". The areas B-I are B-II are the "tails" corresponding to A-I and A-II, respectively. The tails are subtracted from the pulse-height distribution. The area [(A-I)+ (A'-I)] and that of [(A-II)+(A'-II)] are elements of Q(E), where the area of A'-I and A'-II are equal to that of B-I and B-II, respectively.

highest pulse-height element of $\langle P(\varepsilon) \rangle$, the "tails" are subtarcted from the pulse-height distribution and added to the corresponding "peak". This process is carried out in successive steps and a spectrum $\langle Q(E) \rangle$ is obtained as shown schematically in Fig. 1, where E denotes the energy of gamma rays.

For the purpose of mathematical explanation of this method, we have an array numbers a_{ij} in a matrix form. Each column of the matrix represents the unfolding strip, and its "peak" is located on a diagonal element of the matrix. Thus the elements of the $n \times n$ matrix A, a_{ij} , are given in the first approximation as,

$$a_{ij} = r(E_j) \qquad \text{for } i = j$$

= $[1 - r(E_j)]/f(E_j) \qquad \text{for } i \le f(E_j)$
= $0 \qquad \text{for } i \ge f(E_j) \text{ and } i \ne j$ (1)

where $f(E_j)$ refers to the location of the pulse height interval containing the edge of the Compton electron distribution in the usual meaning, and $r(E_j)$, the photo-fraction of the scintillator.

The application of the unfolding method to the convertion of a pulse height distribution $P(\epsilon)$ to a energy spectrum Q(E) is written as,

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$$a_{ij}q_i = p_i - \sum_{k=i+1}^{n} a_{ik}q_k.$$
 (2)

where p_i and q_i represent the elements of $P(\varepsilon)$ and Q(E), respectively. Considering the matrix A is triangular as shown in Eqs. (1),

$$\langle P(\epsilon) \rangle = A \langle Q(E) \rangle$$
 (3)

then

$$\langle Q(E) \rangle = A^{-1} \langle P(\varepsilon) \rangle$$
 (4)

Thus we know that this simple unfolding method is also the simplification of inverse matrix method.

Simulation of Utility by Calculation

A series of numerical calculations has been carried out for the purpose of demonstrating the utility of this method.

Figure 2a shows the "ideal" case of measurement of gamma rays by a scintillation spectrometer and an "ideal" conversion method is applied to pulse-height distribution and resultant spectrum may be identical to the spectrum of the incident gamma rays, N(E). It is difficult to know that the incident gamma ray spectrum strictly, because there is no "ideal" conversion method. A very similar spectrum to that of the incident gamma rays may be obtainable only when geometrical arrangement is free from scattered radiation and incident gamma ray spectrum are very simple. Figure 2b shows the case of the usual unfolding method.



Fig. 2. Schematic illustrations of (a) measurement of gamma rays and "ideal unfolding"; (b) me measurement of gamma rays and unfolding; (c) simulating calculation and application of the simple unfolding method.

The fact mentioned above is the reason that the demonstration of the utility of a new method is not easy. For the purpose of taking away this difficulty, the measurement in Fig. 2b is replaced by the simulating calculation.

The response functions of a scintillator are postulated first for many monochromatic gamma rays. These functions compose a response function matrix M. Multiplicating this matrix to the postulated gamma ray spectrum N(E), and the calculated pulse-height distribution $P(\varepsilon)$ is obtained. Then

$$\langle P(\varepsilon) \rangle = M \langle N(E) \rangle \tag{5}$$

where $\langle P(\varepsilon) \rangle$ and $\langle N(E) \rangle$ are one-column matrices.

This specturm $\langle P(\varepsilon) \rangle$ is converted to the corrected gamma ray specturm $\langle Q(E) \rangle$ by the method described previous paragraph. The utility of this method may be understood by comparing N(E) with Q(E).

The postulated response function matrix M is shown in Table I. It is composed of fifty pulse-height distributions of monochromatec gamma rays with energies from 15 keV to 1485 keV. Each pulse-height distribution has a photopeak being assumed as the Gaussian distribution and a flat Compton electron distribution. The photofractions used in this simulating calculating are shown in Fig. 3 as a function of energy. Curve I shows the photofraction for 8-in. diameter by 4-in. long NaI(Tl) scintillator obtained by experiment and Curve II shows photofraction of scintillator as large as or larger than 8-in. diameter by 8-in. long posturated from the data obtained by Kreger and Brown⁶ and Love and Chapman⁷. The halfwidth of the scintillators are shown in Fig. 4. For the sake of simplicity, the



Fig. 3. The photofractions used in the simulating calculations. The Curve I is the photofraction obtained with 8-in. diameter by 4-in. long NaI(Tl) scintillator and the Curve II is the postulated photofraction for a NaI (Tl) scintillator larger than 8-in. diameter by 8-in. long.



Fig. 4. The postulated half-width used in this calculation.

efficiency of the scintillator was postulated to be equal to unity throughout the region of gamma ray energies involved. The matrix A which is used in this calculation is shown in Table II. The photofraction shown in Fig. 3 was also used for the calculation of the element of Matrix A.

Several typical gamma ray spectra N(E) were assumed. The pluse-hight distributions $P(\epsilon)$ and the corrected spectra Q(E) corresponding to the assumed spectra N(E) were calculated and given in Table III a to c.

These spectra are not corrected for the Gaussian broadening of the photopeak. The correction for this broadening is not simple. For the application to analysis of discrete gamma rays, the determination of incident gamma ray energy and intensity is not as difficult the same usual strip off method.

Example of Application to the Analysis of Gamma Rays

This unfolding method was applied to the analysis of pulse height distributions obtained by an 8-in. in diameter by 4-in. long NaI(Tl) scintillator placed in a



Fig. 5. Shows Examples of application of the simple unfolding method for scintillation spectra obtained with 8-in. diameter by 4-in. long NaI(Tl) scintillator. Gamma ray spectra from a ¹³⁷Cs point isotropic source are shown: (a) from a point source placed 4 cm from the surface of the scintillator; (b) from a point source placed on a 1 cm thick iron plate placed at a distance 50 cm from the surface of the scintillator; (c) from ¹³⁷Cs solution in a phantom with the capacity of 10.86 *l*.





1305 1335 1365 1395 1425 1455 1485	E 5 15 45 75 105 165 225 255 255 255 255 315 315 315 375 405 435 555 585 615 645 645 645 675 705 785 825 825 825 825 825 105 105 105 105 105 105 105 10	\mathbf{X}
	5 <u>15</u> 100000 <u>1000</u>	
	45 100 100	
	75, 00000 100	
	105	
	135 0000 10	
	165 00000 1	
	195 000000 10	
	225	
	255	
	285 74 74 74 30 100000	
	315 296 296 296 296 296 55 100000	
	345 470 470 470 470 470 15 100000	
	375 586 586 586 586 586 586 586 586	
	405 692 692 692 692 692 505	
	435 749 749 749 749 749 749 749 438	
	465 809 809 809 809 809 809 809 809 809 809	
	495 852 852 852 852 852 852 852 852 277	
	525 909 909 909 909 909 909 909 909 909 204	
	555 942 942 942 942 942 942 942 942 942 942	
	585 969 969 969 969 969 969 969 969 969 96	
	6 615 1001	
	i 645 1 1017 1 1017	
	6 678 7 1027 7 1027 807	
	8 7 7 10: 7 10: 7 10: 7 10 1000 7 1000 10000	
	05 7 127 10 127 10 127 10 127 10 127 10 127 10 127 10 127 10 127 10 127 10 127 10 127 10 107 10 107 10 107 10 107 10 107 10 107 10 107 10 107 10 107 10 107 10 107 10 1007 10 1000 10000	
	735 144 11 144 12 144 14 144 14 146 14 1	
	765 1046 1046 1046 1046 1046 1046 1046 1046	
	795 1055 1055 1055 1055 1055 1055 1055 10	
	825 1066 1066 1066 1066 1066 1066 1066 106	
	855 1073 1	
	885 1080 1	
	915 1054 1084 1084 1084 1084 1084 1084 1084 108	
	945 1089 1	
	975 1094 1004 1	
	1005 1099	
	1035 1104	
	1065 1102	
	1095 1103 1103 1103 1103 1103 1103 1103 110	
	1125 1103	
	1155 1099 1099 1099 1099 1099 1099 1099	
	1185 1098 1088	
	1215 1100 1100 1100 1100 1100 1100 1100	
	12455 1097	
	1275 1100 1100 1100 1100 1100 1100 1100 11	
	1305 1097	
100000	1335 1101	
100000	1365 1100	
100000	1395 1099	
100000	1425 1097	
100000	1455 1096 1006 1006 1006 1006 1006 1006 1006 1006 1006 1006 1006 1006	
100000	1485 1091	

Table I. The postulated response function matrix, M. The efficiency of the scintillators for gamma-rays was postulated to be equal to unity. The width of each interval (keV), to yield the matrix elements the number in the table should be multiplied by 10⁻⁴.

795	825	855	885	915	945	975	1005	1035	1065	1095	1125	1155	1185	1215	1245	1275	1305	1335	1365	1395	1425	1455	1485
						050		0.4.2	0.24	0.00	`			•									
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	005	600	054	050	040	042	034	020	022	013	806	801	793	789	781	///	773	765	758	752	744
867	868	865	005	033	054	850	040	842	834	828	022	013	806	001	793	789	781	///	773	765	758	752	744
867	868	000	805	633	954	850	940	842	934	929	042	013	806	801	793	789	781	///	773	765	758	752	744
867	808	800	865	955	854	850	846	842	834	828	822	812	806	801	793	789	781	///	773	765	758	752	744
007	000	005	945	055	854	850	846	842	834	828	822	812	806	801	793	769	781	///	//3	765	758	752	744
867	000	005	005	055	854	850	846	842	834	828	922	912	806	801	793	769	781	///	773	765	758	752	744
867	808	865	865	855	854	850	846	842	834	828	822	813	800	801	793	709	781	777	//3	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	701	777	773	765	758	/52	744
967	969	865	865	855	854	850	846	842	834	828	822	813	806	801	793	799	701	777	773	765	/58	/52	/44
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	750	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	750	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
867	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
460	868	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
4	393	865	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
101	6	370	865	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
1176	128	8	305	855	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
6774	1321	157	12	280	854	850	846	842	834	828	822	813	806	801	793	789	781	777	773	765	758	752	744
19337	6996	1462	192	16	250	850	846	842	834	828	822	. 813	806	801	793	789	781	777	773	765	758	752	744
27415	19001	7185	1612	229	22	200	150	842	834	828	822	. 813	806	801	793	789	781	777	773	765	758	752	744
19337	26495	18665	/366	1/58	1000	20	100	134	834	020	822	013	806	801	793	789	781	777	773	765	758	752	744
6774	19001	25645	16310	10020	7645	2025	350	42	116	828	022	.013	806	801	793	789	781	777	773	765	758	752	744
11/6	1221	2105	10216	24002	17697	7761	2147	395	50	65	822	813	806	801	793	789	781	777	773	765	758	752	744
101	1321	1462	7366	18020	23384	17370	7846	2271	438	64	25	797	806	801	793	700	701	777	773	765	758	752	744
-	. 120	157	1612	7531	17687	22714	17068	7925	2382	511	76	8	200	801	793	705	701		773	765	/58	752	744
		±27	192	1758	7645	17370	22108	16760	7985	2560	563	87	11	767	793	789	701	272	773	765	758	/52	744
		ŭ	12	229	1888	7761	17068	21508	16493	8097	2674	609	102	14	738	789	781	777	773	765	750	752	744
				16	266	2025	7846	16760	20999	16150	8142	2771	669	123	16	686	781	777	773	765	750	752	744
					21	309	2147	7925	16493	20326	15874	8175	2887	744	136	21	636	777	773	765	758	752	744
					1	27	350	2271	7985	16150	19827	15642	8208	3028	789	162	24	658	773	765	758	752	744
						1	34	395	2382	8097	15874	19415	15360	8250	3101	873	178	27	629	765	758	752	744
							1	42	438	2560	8142	15642	18925	15049	8251	3242	923	211	34	570	758	752	744
								3	50	511	2674	8175	15360	18385	14837	8277	3315	1017	226	43	587	752	744
									2	64	563	2771	8208	15049	18040	14519	8269	3454	1054	260	52	575	744
										5	76	609	2887	8250	14837	17508	14304	8278	3496	1146	298	58	567
											7	87	669	3028	8251	14519	17170	13981	8239	3629	1234	317	68
												8	102	744	3101	8277	14303	16649	13774	8271	3734	1277	354
													· 11	123	789	3242	8269	13981	16347	13555	8237	3777	1358
														14	136	873	3315	8278	13774	15981	13236	8199	3871
															16	162	923	3454	8239	13555	15503	13048	8182
																21	178	1017	3496	8271	13236	15233	12815
																2	24	211	1054	3629	8237	13048	14883

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Medium energy		.8″ø	×4″	Larger than $8''\phi \times 8''$		
(keV)	N(E)	$P(\varepsilon)$	Q(E)	$P(\varepsilon)$	Q(E)	
15	0	68	0	40	0	
45	0	68	0	40	0	
75	0	68	Ō	40	0	
105	õ	68	õ	40	0	
135	õ	68	ŏ	40	0	
165	Ő	68	ŏ	40	Ō	
195	õ	68	ŏ	40	Ő	
225	õ	68	ŏ	40	Ō	
255	Ő	68	ŏ	40	0	
285	õ	68	Ő	40	Ő	
315	õ	68	õ	40	õ	
345	Ő	68	Ő	40	Ő	
375	ň	68	ñ	40	õ	
405	Ő	68	ů N	40	õ	
435	0	68	0	40	õ	
465	0	68	ů	40	0	
495	0	68	0	40	0	
595	0	60	0	40	0	
555	0	60	. 0	40	0	
595	0	00	0	40	0	
615	0	60	0	40	0	
645	0	00	0	40	0	
043	0	08	0	40	0	
075	0	00	0	40	0	
703	0	08 60	0	40	0	
7 3 0	0	68	0	40	0	
/03	0	68	0	40	0	
y95	0	68	0	40	0	
825	0	68	1	40	0	
800	0	68	0	40	0	
885	0	68	0	40	0	
915	0	68	0	40	2	
945	U	68	5	40	8	
975	0	68	43	40	20	
1005	0	0	-43	0	2	
1035	0	4	-12	5	-/	
1065	0	25	38	33	4	
1095	0	108	192	144	190	
1125	0	308	549	492	546	
1155	0	575	1054	768	1031	
1185	5000	709	1288	946	1279	
1215	0	575	1065	768	1046	
1245	0	308	569	492	564	
1275	0	108	204	144	200	
1305	0	25	47	33	48	
1335	0	4	8	5	7	
1365	0	0	0	0	0	
1395	0	0	0	0	0	
1425	0	0	0	0	0	
1455	0	0	0	0	0	
1485	0	0	0	0	0	

Table III (a). The results of a series of simulating calculations. N(E) are assumed to be incident gamma ray spectra, $P(\varepsilon)$ are calculated pulse height distribution, Q(E) are response corrected spe ctra.

Medium energy	N(F)	8″¢	×4″	Larger than $8''\phi \times 8''$		
(keV)	14 (12)	$P(\varepsilon)$	Q(E)	$P(\varepsilon)$	Q(E)	
15	5000	8402	5074	6577	5028	
45	5000	8402	5074	6577	5028	
75	5000	8402	5074	7005	5456	
105	5000	8402	5074	6261	4720	
135	5000	8402	5074	6575	5026	
165	5000	8351	5065	6582	5032	
195	5000	8276	5049	6555	5050	
225	5000	9079	5937	6535	5064	
255	5000	6905	4171	6494	5062	
285	5000	6432	3887	6433	5062	
315	5000	7179	4987	6333	5077	
315	5000	7179	4987	6333	5077	
345	5000	6932	4995	6217	5076	
375	5000	6722	4088	6116	5083	
405	5000	6516	4083	6008	5086	
435	5000	6322	4002	5012	5045	
465	5000	6014	4083	5800	5013	
495	5000	5813	4082	5606	5027	
525	5000	5632	5006	5596	5034	
555	5000	5740	5028	5496	5031	
585	5000	5318	5015	5405	5039	
615	5000	5158	5060	5811	5041	
645	5000	4000	5031	5208	5035	
675	5000	4866	5030	5189	5035	
705	5000	4744	5055	5058	5047	
735	5000	4610	5053	4071	5038	
765	5000	4475	5048	4881	5025	
795	5000	4347	5052	4792	5020	
825	5000	4233	5074	4708	5020	
855	5000	4125	5023	4627	5025	
885	5000	4013	5040	4546	5028	
915	4000	3903	5062	4465	5020	
945	5000	3794	5086	4386	5024	
975	5000	3621	5033	4281	4989	
1005	5000	3526	5075	4237	5034	
1035	5000	3434	5034	4117	5046	
1065	5000	3338	5069	4091	5042	
1095	5000	3239	5101	4013	5034	
1125	5000	3145	5048	3946	5035	
1155	5000	3064	5106	3882	5047	
1185	5000	2975	5055	3814	5041	
1215	5000	2881	5082	3748	5040	
1245	5000	2805	5043	3681	5030	
1275	5000	2791	5067	3616	5020	
1305	5000	2640	4980	3650	5000	
1335	5000	2600	4999	3530	5000	
1365	5000	2584	4969	3485	4978	
1395	5000	2511	4828	3382	4860	
1425	5000	2304	4589	3154	4563	
1455	5000	2011	3942	2713	3950	
1495	5000	1521	2042	2054	3012	

Table III (b)

Table III (c)

Medium energy of interval	N(F)	8″	\$\phi \times 4''	Large 9″ø	r than ×8″
(keV)	11(E)	$P(\varepsilon)$	Q(E)	$P(\epsilon)$	Q(E)
a 15 a a	4900	6418	4894	5499	4910
45	4800	6318	4794	5399	4801
75	4700	6218	4694	5693	4501
105	4600	6118	4594	4897	4301
135	4500	6018	4494	5092	4498
165	4400	5817	4327	4997	4413
195	4300	5717	4286	4889	4327
225	4200	6279	4914	4762	4224
255	4100	4391	3352	4632	4121
285	4000	4686	3911	4486	4020
315	3900	4478	3897	4313	3030
345	3800	4212	3805	4130	3827
375	3700	3980	3600	3063	3731
405	3600	3757	3507	3704	3631
435	3500	3549	2502	3734	3031
465	3400	2077	3 3304	9479	3300
495	3300	2070	2020	3474 9916	3397
525	3300	3076	3200	9166	3293
555	3200	2897	3208	3166	3197
500	3100	2/33	3092	3019	3094
565	3000	2579	2984	2880	2998
615	2900	2425	2909	2743	2899
645	2800	2275	2792	2604	2795
6/5	2700	2144	2694	2481	2696
705	2600	2021	2602	2363	2601
735	2500	1896	2500	2239	2501
765	2400	1774	2396	2117	2394
795	2300	1659	2296	1998	2292
825	2200	1554	2204	1884	2192
855	2100	1454	2097	1774	2093
885	2000	1355	1989	1666	1994
915	1900	1259	1896	1561	1891
945	1800	1167	1803	1459	1793
975	1700	1065	1683	1354	1694
1005	1600	984	1595	1263	1595
1035	1500	906	1482	1170	1497
1065	1400	829	1392	1078	1396
1095	1300	755	1300	987	1293
1125	1200	683	1185	900	1193
1155	1100	617	1098	816	1096
1185	1000	547	979	734	994
1215	900	487	893	653	894
1245	800	429	788	574	793
1275	700	369	695	497	693
1305	600	313	590	499	503
1335	600	258	496	349	403
1365	400	200	304	976	204
1395	300	153	204	206	397
1425	200	105	234	149	290
1455	200	105	200	144	200
1405	100	04	99	8/	120
1403	U	34	08	40	67

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steel iron box lined by 3-mm thick lead plate*. The photofraction of the scintillator is shown in Fig. 3 as Curve I. Figure 5 shows spectra of gamma rays from ¹³⁷Cs point source placed at a distance 4-cm from the face of the scintillator without scatterer and from the source placed on a 1-cm thick iron plate placed at a distance 50-cm from the face of the scintillator, and ¹³⁷Cs solution in a phantom⁸) with the capacity of 10.86*l*. They might be considered response corrected spectra.

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